

DESIGN OF A FPGA-BASED DIGITAL REACTIVITY METER FOR DALAT NUCLEAR RESEARCH REACTOR

Vo Van Tai, Nguyen Van Kien, Le Van Diep, Nguyen Nhi Dien

Dalat Nuclear Research Institute

01 Nguyen Tu Luc, Dalat city, Vietnam

Email: taivnchn@gmail.com

Presenter: Mr. Võ Văn Tài Đà lạt, 10/Dec/2021 1

VINANST-14

9-10 December 2021, Da Lat City, Lam Dong, Vietnam

INTRODUCTION

CONTENTS

1

2

EXPERIMENTS, RESULTS AND DISCUSSION

VINANST-13

REACTIVITY CALCULATION

CONCLUSIONS

4

VINANST-1

I. INTRODUCTION

- Reactor Control system - ASUZ-14R

I. INTRODUCTION

Psr (10⁻⁶ to 10⁻¹ % Pnominal, Pnominal = 500 kWt));

Reactivity is an important parameter that indicates a departure of a reactor power from critical, to calculate whether the neutron density in a reactor will remain constant or change. The reactor is in critical state or operates at the steady power level when $k_{\text{eff}} = 1.0$.

If k_{eff} < 1.0, the reactor is subcritical or operates at the power level is falling.

If k_{eff} > 1.0, the reactor is supercritical or the power level is rising. Reactivity is given by equation (2) [1, 4] as:

$$
\rho = (k_{\text{eff}} - 1) / k_{\text{eff}} \tag{2}
$$

where, ρ is reactivity and k_{eff} is effective multiplication factor of the reactor.

The time behavior of the reactor power level in a nuclear reactor by changes k_{eff} can be expressed by the point kinetics equations are shown in equations (3) and (4) $[1, 4, 5]$ as:

$$
\frac{dP(t)}{dt} = \frac{\rho(t) - \beta}{\Lambda} P(t) + \sum_{i=1}^{6} \lambda_i C_i(t)
$$
(3)

$$
\frac{dC_i(t)}{dt} = \frac{\beta_i}{\Lambda} P(t) - \lambda_i C_i(t) \text{ for } i = 1, 2, ..., 6.
$$
(4)

where,

- $P(t)$ is the reactor power at time t, $\overline{}$
- $\rho(t) = \frac{k(t)-1}{k(t)}$ is the reactor reactivity at time t,
- $Ci(t)$ is the concentration of the i-th group delayed neutron precursors, λi is the decay constant of the i-th group;
- β is the fraction of the delayed neutron for the i-th group; $\overline{}$
- $\Lambda = 1/k$ is the mean neutron generation time. $\overline{}$
- Note that a neutron source was ignored in equation (3). $\overline{}$

- Solution of the equations (3) and (4) we have equation (5) .
- The equation (5) is used to develop the algorithm for the reactivity calculation of \bullet the designed digital reactivity meter module for the DNRR.

•
$$
\rho_n = \frac{\beta_{eff}}{1 + l\tau} + \frac{l\tau}{1 + l\tau} - \frac{1}{P_m(1 + l\tau)} \sum_{i=1}^6 \lambda_i \beta_i S_{im}
$$
(5)

where,

•
$$
\tau = \frac{P_m - P_{m-1}}{P_m \Delta t}
$$
 (6)

•
$$
S_{im} = S_{im-1}e^{-\lambda_i \Delta t} + \frac{1}{\lambda_i} \left(1 - e^{-\lambda_i \Delta t}\right) \left[P_{m-1} - \frac{P_m - P_{m-1}}{\lambda_i \Delta t}\right] + \frac{P_m - P_{m-1}}{\lambda_i} \tag{7}
$$

- where, ٠
- $P_m = P(t), P_{m-1} = P(t \Delta t),$
- Δt is the sampling rate (in seconds), -
- β_{eff} is the delayed neutron fraction of DNRR = 7.464.10⁻³, -
- and S_{im} is the reactor power history. -

The initial value S_{i0} that is the mean power level of the reactor at critical state, is \bullet given by equation (8):

$$
S_{i0} = \frac{P_0}{\lambda_i} \tag{8}
$$

An initial reactivity is equal zero. Kinetic parameters, and of the DNRR are \bullet shown in Table 1.

 $\boldsymbol{B}_{\!\!if\!f}$

• Functions of each unit in the diagram of Fig. 1 are summarized as: Fs is the output pulse frequency from the pulse amplifier, which is proportional to the reactor power; SAMPLE (N(t)) is for sampling pulse frequency Fs; FILTER is for filtering sampled pulses by using moving average technique; S_{1m} to S_{6m} are reactor power history for six groups of delayed neutrons; Microblaze is a microcontroller with 50- MHz on-board clock and 128 kbytes RAM for reactivity calculation; and PC is a computer for recording reactivity.

- The output frequency from a pulse amplifier is sampled and filtered by hardware on FPGA Artix-7.
- In which, the current reactor powers of P_m and P_{m-1} as the reactor power history are determinated, and S_0 and S_{im} (S_{1m} S_{6m}) are also calculated using Eq. (8) and Eq. (7), respectively.
- The Microblaze calculates the reactor reactivity by Eq. (5) and sends its value to the on-board display LCD and to the PC for recording.

• Flow chart of reactivity calculation in FPGA Artix-7 with embedded Microblaze.

III. EXPERIMENTS, RESULTS AND DISCUSSION

• The designed reactivity meter module was tested for its comparison with the imported PNO-121R5 module using the pulse-generated simulator PGT-17R

III. EXPERIMENTS, RESULTS AND DISCUSSION (tt)

12

III. EXPERIMENTS, RESULTS AND DISCUSSION (tt)

III. EXPERIMENTS, RESULTS AND DISCUSSION (tt)

• Figure 4 shows the measured reactivity with the reactor power changes from 4.10⁻⁵ % to 2.10⁻¹% at T = 80s, -80s, 120s and -120s. The results obtained by the designed module and the PNO-121R5 module are equivalent with the discrepancy about 10%. Some experimental results are equivalent as shown in Table 2.

IV. CONCLUSION

• A digital reactivity meter module was designed and constructed. The output pulses of the amplifier, which are proportional to the reactor power, are sampled and filtered by hardware in FPGA.

• The tasks for the solution of the point reactor kinetics equations to calculate reactivity are performed in parallel on the FPGA hardware and embedded Microblaze.

• The experimental results of measured reactivity by the FPGA-based module that was compared with that by the PNO-121R5 show that the designed reactivity meter module can be used to measure and monitor the reactivity of the DNRR.

• The obtained results are only the first step for next experiments to the measurement of effective reactivity of the control rods including compensation rods and automatic regulation rod of at the DNRR basing on the rod drop method.

VI. REFERENCES

- [1] S. E. Binneyand A. J. M. Bakir, "Design and Development of a Personal—Computerbased Reactivity Meter for a Research Reactor", Nuclear Technology, Vol. 85 APR, Oregon State University, Corvallis, 1989, pp. 97331-105902.
- [2] Buzzetti S., Capou M., Guazzoni C. et al., "High-speed FPGA based pulse-height analyzer for high resolution X-ray spectroscopy", IEEE Trans. Nucl. Sci. 52, pp. 854–860, 2005.
- [3] Complex of Equipment for Control and Protection System ASUZ-14R, Operating Manual RUNK.506319.004 RE-E, JSC SNIIP SYSTEMATOM, Chief Designer А. А. Zaikin, 2006.
- [4] Auerbach, J. M. and S. G. Carpenter (1978), "A Microprocessor Controlled Reactivity Meter for Real Time Monitoring of Reactors", Nuclear Science, IEEE Transactions on 25(1): 98-100.
- [5] Ansari, S. A. (1991), "Development of On-line Reactivity Meter for Nuclear Reactors", IEEE Transactions on Nuclear Science 38(4): 946-952.
- [6] Nguyen Nhi Dien, "Safety Analysis Report for the Dalat Nuclear Research Reactor, Dalat Nuclear Research Institute", 2012.
- [7] International Atomic Energy Agency, "Hands-on Training Courses Using Research Rectors and Accelerators", series 57, pp. 15-17, Vienna, (2014).

Thank you for your attention!